

## Introduction

- RAPID** (Real-time Analysis for Particle transport and In-situ Detection)
- Solves radiation transport problems using the MRT approach
  - Solves the Fission Matrix equations to obtain  $k_{eff}$ , M, & 3D Fission Density
- Benchmarking performed on the USNA Subcritical Reactor (USNA-SCR)

## Fission Matrix Methodology

FM equations are obtained by recasting the Linear Boltzmann Equation.

Eigenvalue:

$$F_i = \frac{1}{k} \sum_{j=1}^N a_{ij} F_j$$

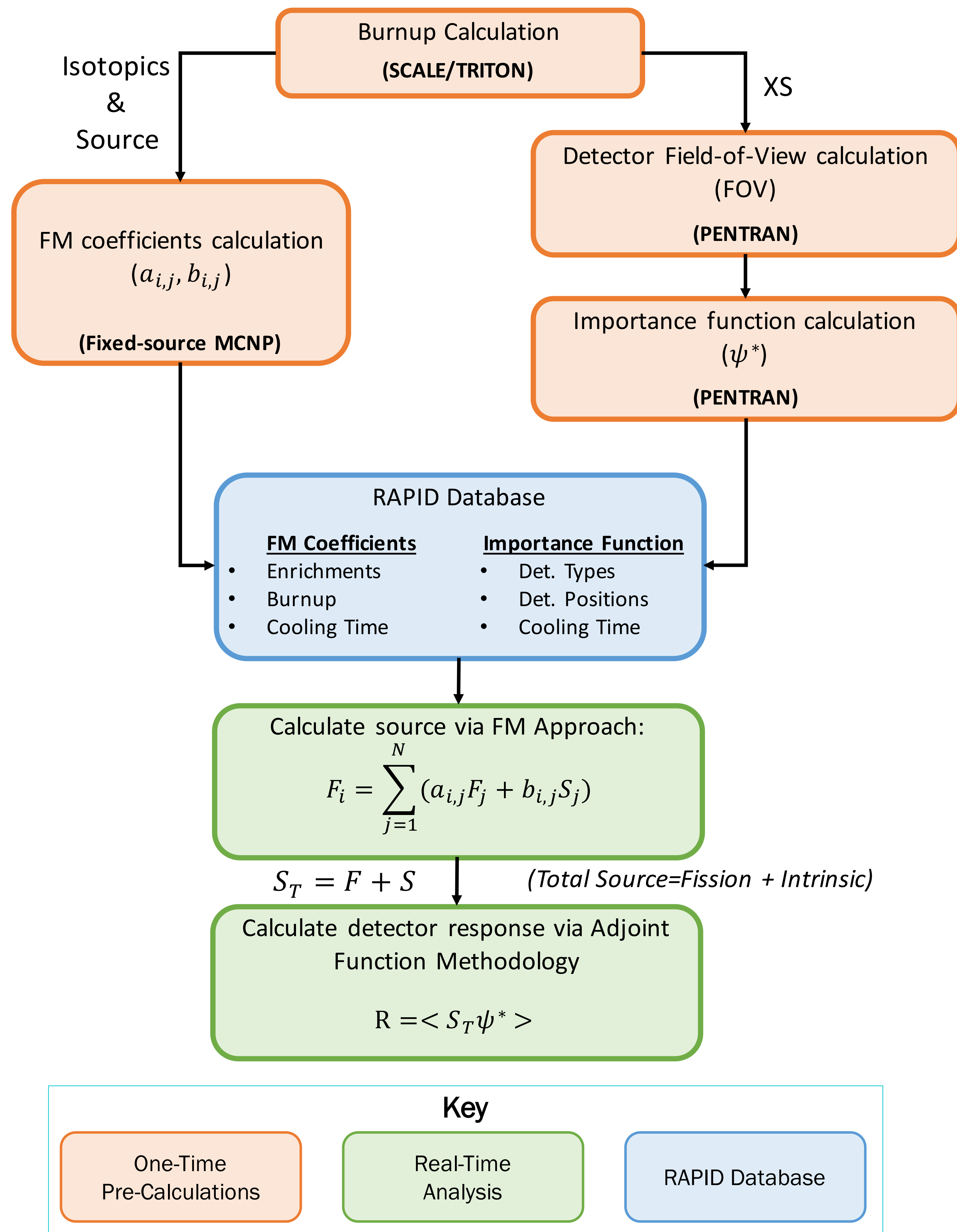
$F_j$  = fission neutron source  
 $S_j$  = intrinsic neutron source

Subcritical Multiplication:

$$F_i = \sum_{j=1}^N a_{ij} F_j + b_{ij} S_j$$

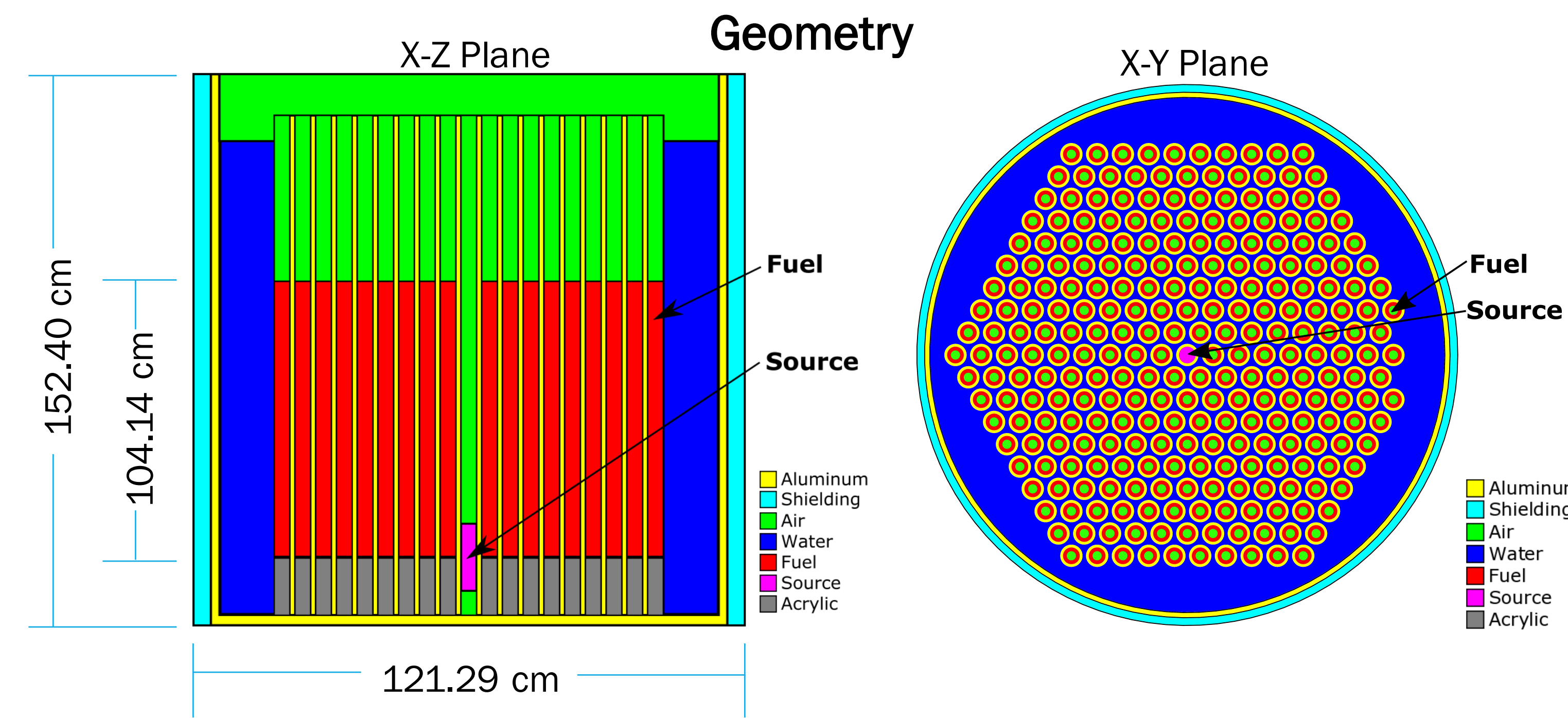
$a_{ij}$  &  $b_{ij}$  = FM coefficients  
 $N$  = number of cells

## RAPID Code Flowchart using the MRT Methodology



## USNA-SCR Geometry

- Light-water moderator
- Natural uranium fuel, coated with aluminum
- PuBe neutron source
- 268 annular fuel rods

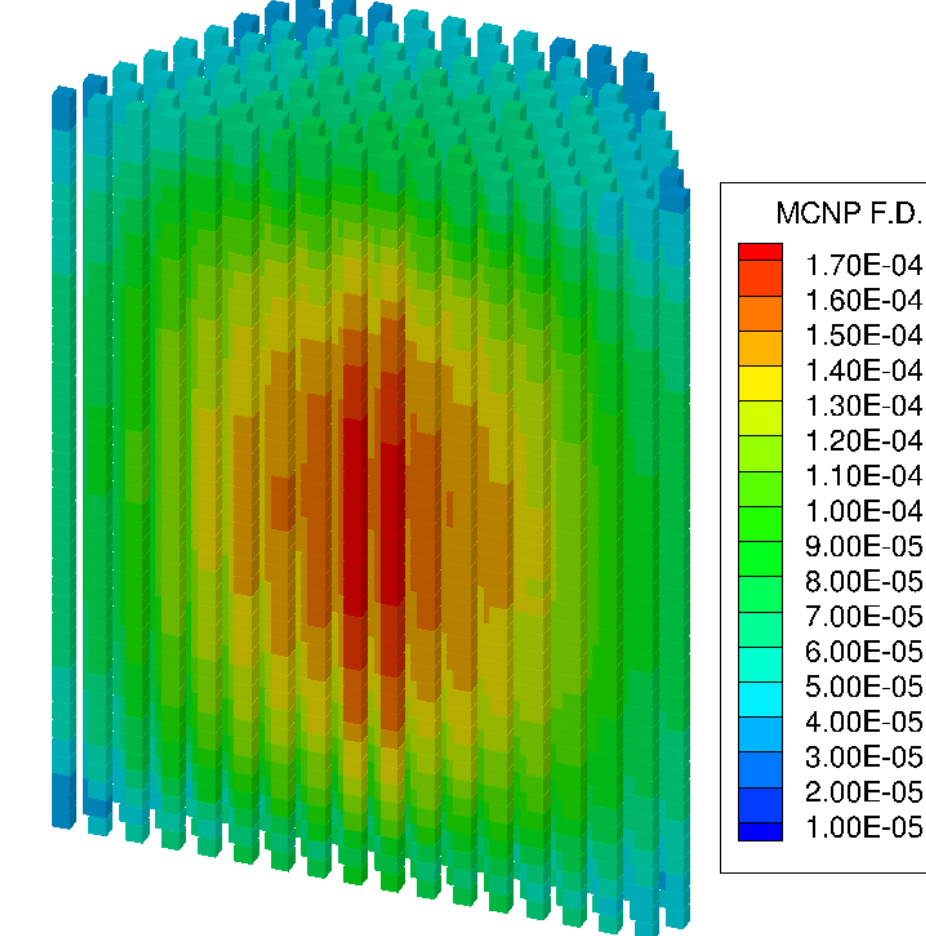


## RAPID vs. MCNP

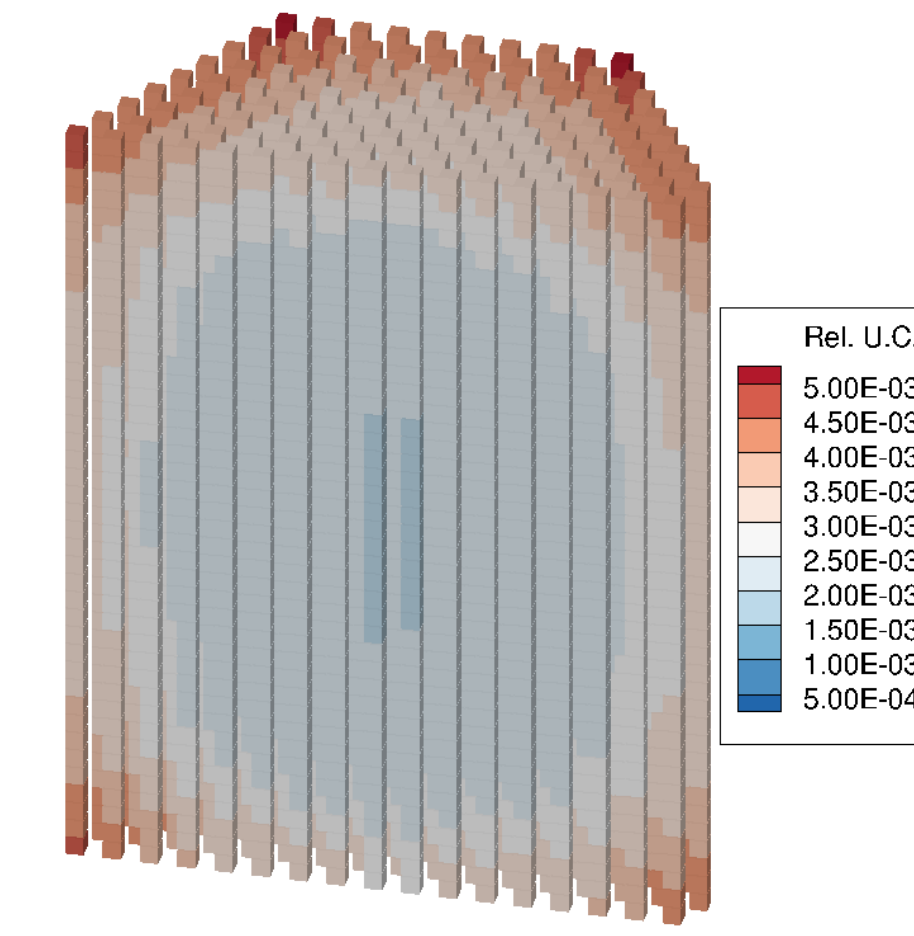
| Eigenvalue |                      |                  |                  | Subcritical Multiplication |                      |                |                   |
|------------|----------------------|------------------|------------------|----------------------------|----------------------|----------------|-------------------|
| Code       | k                    | Rel. Diff. [pcm] | Time [min]       | Code                       | M                    | Rel. Diff. [%] | Time [min]        |
| MCNP       | 0.87278 <sup>1</sup> | -                | 876 <sup>2</sup> | MCNP                       | 6.62042 <sup>1</sup> | -              | 1134 <sup>2</sup> |
| RAPID      | 0.87420              | 163              | 0.7 <sup>3</sup> | RAPID                      | 6.54819              | 1.09           | 0.7 <sup>3</sup>  |

<sup>1</sup>1σ = 3 pcm; <sup>2</sup>On 16 processors; <sup>3</sup>On 1 processor & Speedup = 1251

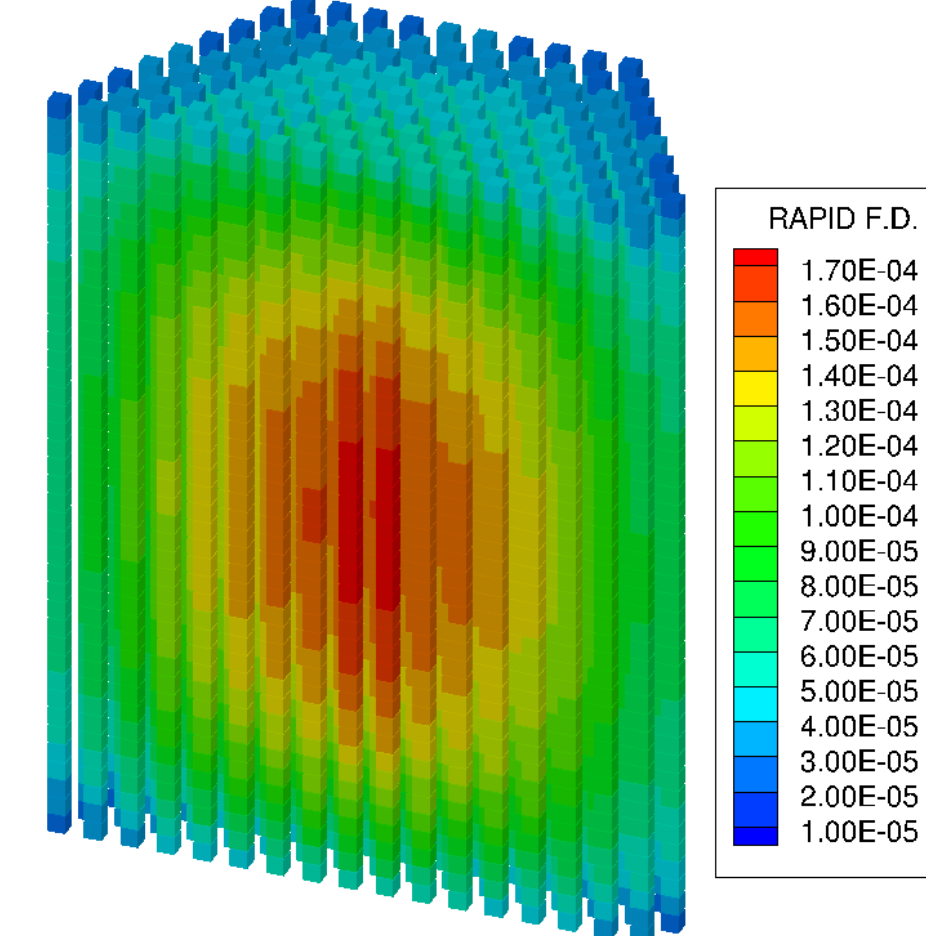
MCNP Subcritical Multiplication Source



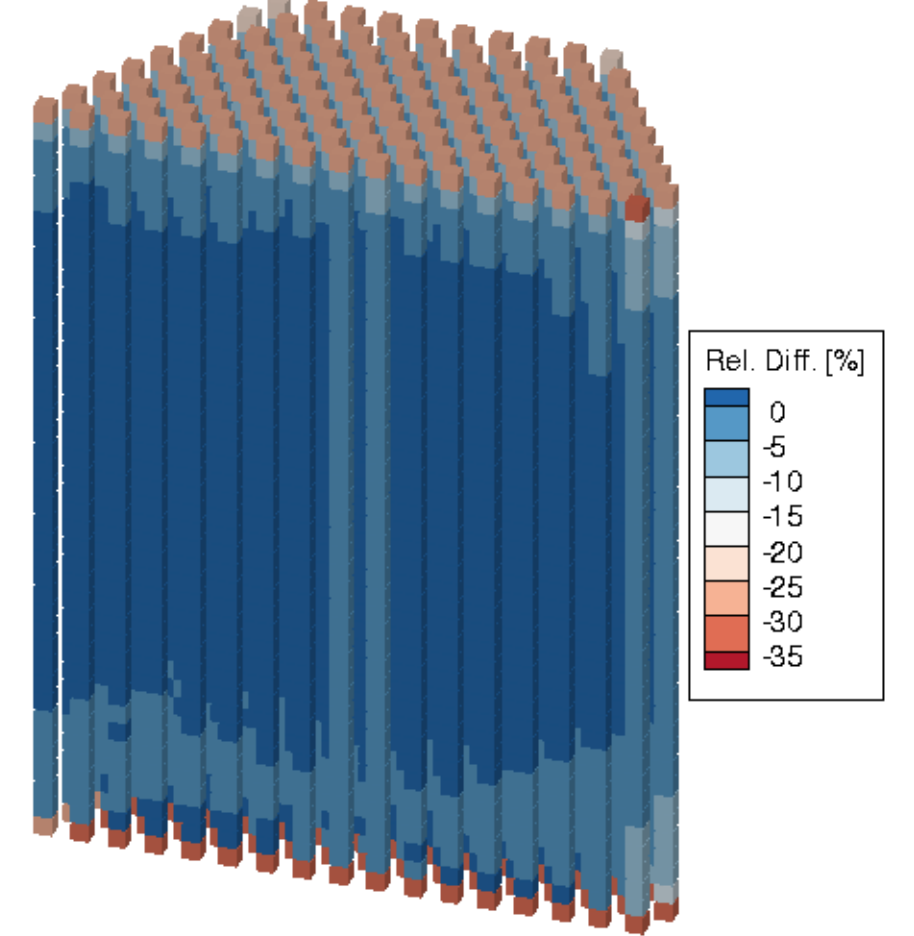
MCNP Tally 1σ Uncertainty



RAPID Subcritical Multiplication Source



Relative Differences vs. MCNP Reference Calculation



## Detectors – Experimental Benchmarking

| Fill Gas        | Pressure | Dimension            | Use     |
|-----------------|----------|----------------------|---------|
| <sup>3</sup> He | 0.41 MPa | H=2.72 cm D=0.97 cm  | In-Core |
| <sup>3</sup> He | 1.01 MPa | H=76.20 cm D=2.54 cm | Ex-Core |

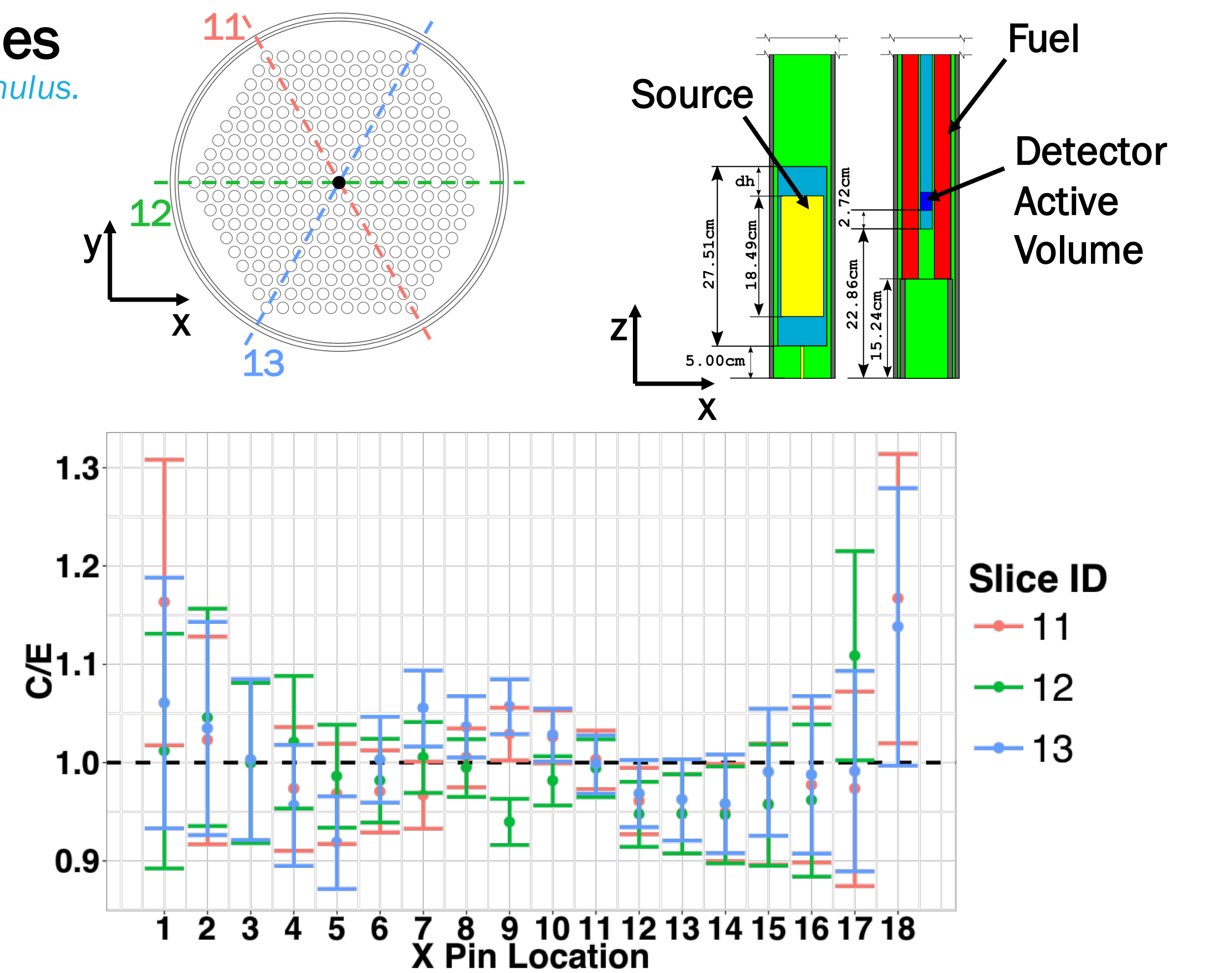
- Neutron Pulse Monitor with 128 Energy Discrimination Bins used for data collection
- Cutoff Bin at observed valley (between gamma noise and neutron counts)

## Calculated vs. Experimental Detector Responses

### In-Core : Radial Profiles

Small <sup>3</sup>He detector within fuel annulus.

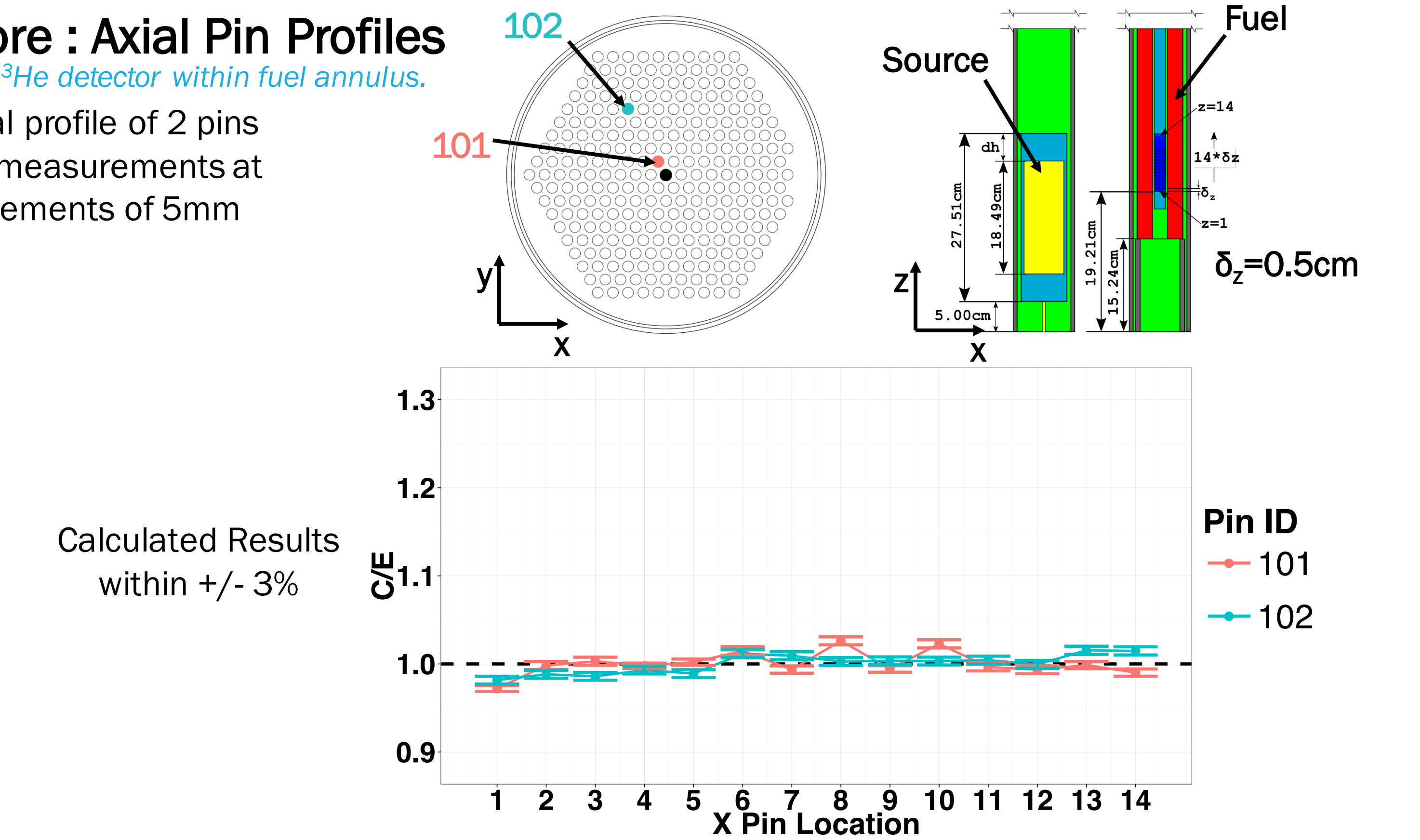
- 3 radial profiles measured along diagonals
- Detector at a fixed axial position



### In-Core : Axial Pin Profiles

Small <sup>3</sup>He detector within fuel annulus.

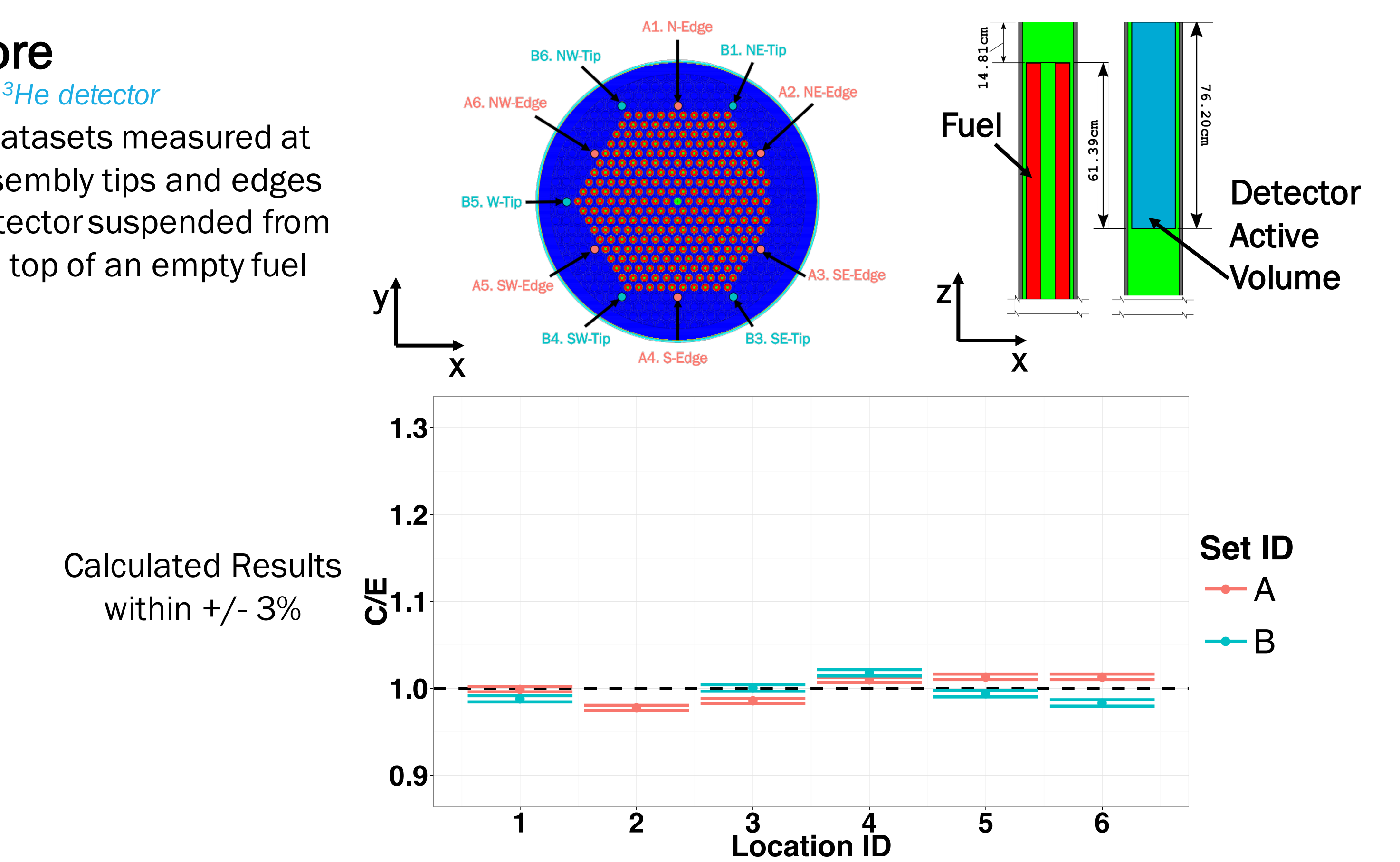
- Axial profile of 2 pins
- 14 measurements at increments of 5mm



### Ex-Core

Large <sup>3</sup>He detector

- 2 datasets measured at assembly tips and edges
- Detector suspended from the top of an empty fuel rod



## Conclusions

This paper demonstrated through computational and experimental benchmarking that RAPID calculates accurate solutions in real-time.